# 22.106 Neutron Interactions and Applications (Spring 2005)

# Problem Set No. 4

# Due: April 7, 2005

# Problem 1

Write down the Neutron Transport Equation defining all the terms in the equation. Derive the streaming term in the Transport equation.

#### Problem 2

Derive the escape probability  $P(E' \rightarrow E)$ . Discuss how you would calculate this quantity using MCNP. Discuss the conditions under which you would or would not expect the MCNP calculation to agree with a numerical evaluation of your analytical expression.

## Problem 3

Consider one-dimensional neutron transport in a purely scattering medium where the cross section  $\sigma(x)$  is a known function of position. Apply the same kind of argument used for Problem 2 to derive an expression for the probability  $P(x' \rightarrow x)$  that a neutron will go the distance from x' to x without scattering, where x > x' but otherwise both are arbitrary. Discuss your result as compared to  $P(E' \rightarrow E)$ 

#### Problem 4

Define the variable lethargy u and show that the average increase in lethargy per collision is given by

$$\xi = 1 + \frac{\alpha l \ell n \alpha}{1 - \alpha} \sim 2/A \text{ for A} >>1$$

where  $\alpha = \left[ (A-1)^2 / (A+1)^2 \right]$ . Discuss how  $\xi$  can be used to estimate the collision density in a medium where A > 1.